



# ACTINIDE SOLUBILITY AND SPECIATION IN THE WIPP TRANSURANIC REPOSITORY

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and the ANS Trinity Chapter



# Overview

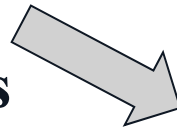
- **Repository Science Directions in the US**
- **WIPP Overview and Status**
- **Actinide Solubility - Research Status and License Approach**
  - **Arguments for the reduction of higher-valent actinides**
  - **Solubility of An(III) – Nd(III) in Brine**
  - **Solubility of An(IV) - Th(IV) and Pu(IV)**
  - **Solubility of An(V) – Inventory limits**
  - **Solubility of An(VI) – Uranium(VI) in brine**
- **Summary of Observations**



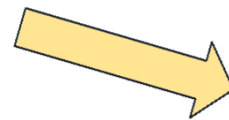
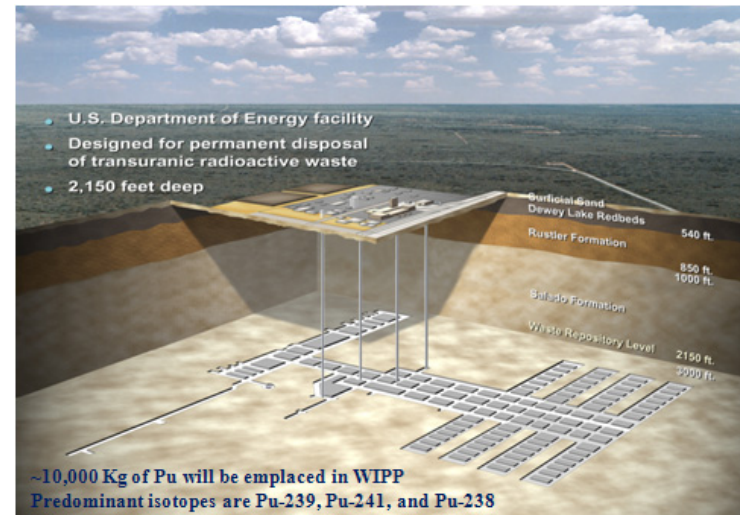
# Repository Science: Strategic Elements

## Key Concepts for the Geologic Disposal of Nuclear Waste

- Geologic isolation
- Favorable thermodynamics and chemistry
  - Reducing conditions
  - Reactive redox control
  - Mildly alkaline pH
- Cost is an issue
- Transparent process with good societal receptiveness is very critical



WIPP – meets these criteria



**Nuclear HLW waste repositories in the US and internationally?**



# Status of Repository Science in the US

- **WIPP continues forward**
- **Yucca Mountain Project – currently no path forward**
- **The Blue Ribbon Commission on America’s Nuclear Future has recommended a path forward**
- **Responsibility for managing used fuel and HLW waste lies with DOE Office of Nuclear Energy (NE), they are to study various options for geologic disposal**
  - **Salt, crystalline (granites) and sedimentary (clay/shale) rocks**
  - **Repositories and/or deep boreholes**
  - **International cooperation is a key to progress**



# Status of Salt-based Repository Concepts

- Second recertification of the WIPP TRU repository received on **November 18, 2011**. The third recertification application will be submitted in **March 2014**
- Blue ribbon commission visit to Carlsbad and tour of WIPP in **January 2011** – local support continues
- **NM State-level support now exists for expanded/additional mission in salt in SE New Mexico**
  - NM senate letter to the blue ribbon commission supporting other potential missions (**first session 2011**)
  - NM house of representatives letter to the blue ribbon commission recommending additional missions including interim storage (**first session 2011**)
- Alternative options for the salt repository concept are under discussion



# WIPP Background

**WIPP is a permanent disposal facility for TRU waste**

- Located in southeast New Mexico
- First certified in 1998, first shipment in 1999, recertified every 5 years
- Operated by U. S. Department of Energy (DOE)

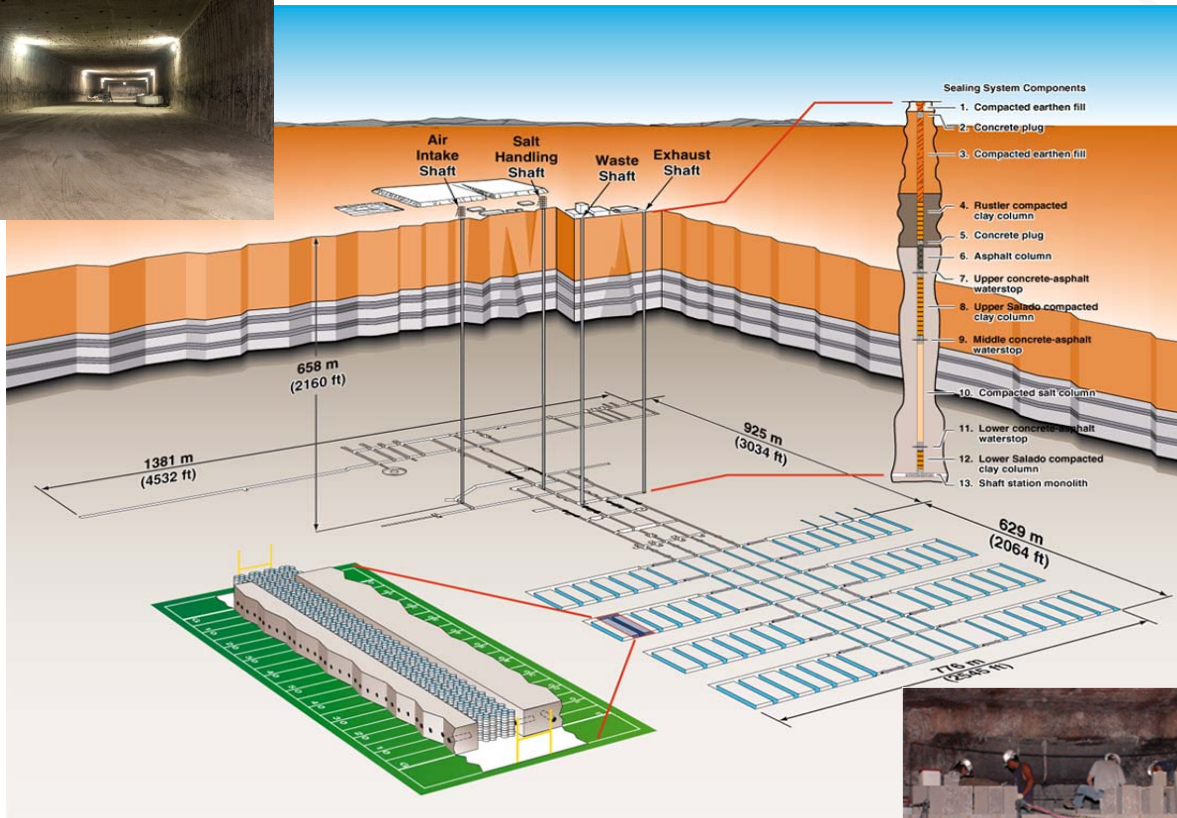


**Regulated by U. S. Environmental Protection Agency (EPA) and New Mexico ED**





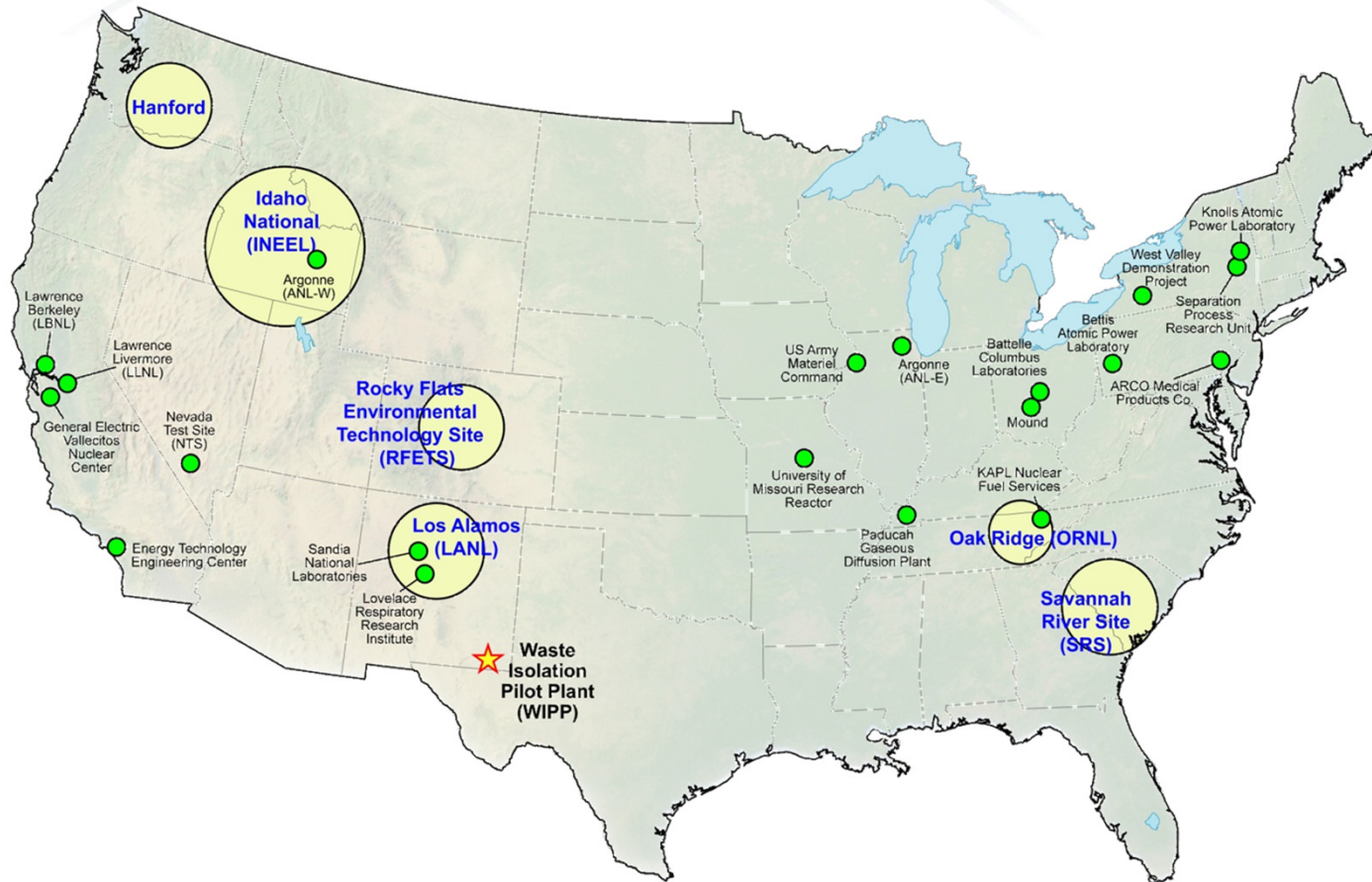
# WIPP Design Overview



**Repository is at a depth of 655 m in the Salado formation**



# Location of the Waste Generator Sites







# Types of Waste

## Contact-handled (CH)

- Large volume
  - ~169,200 m<sup>3</sup> capacity
- No shielding required
- Stacked on floor of waste room



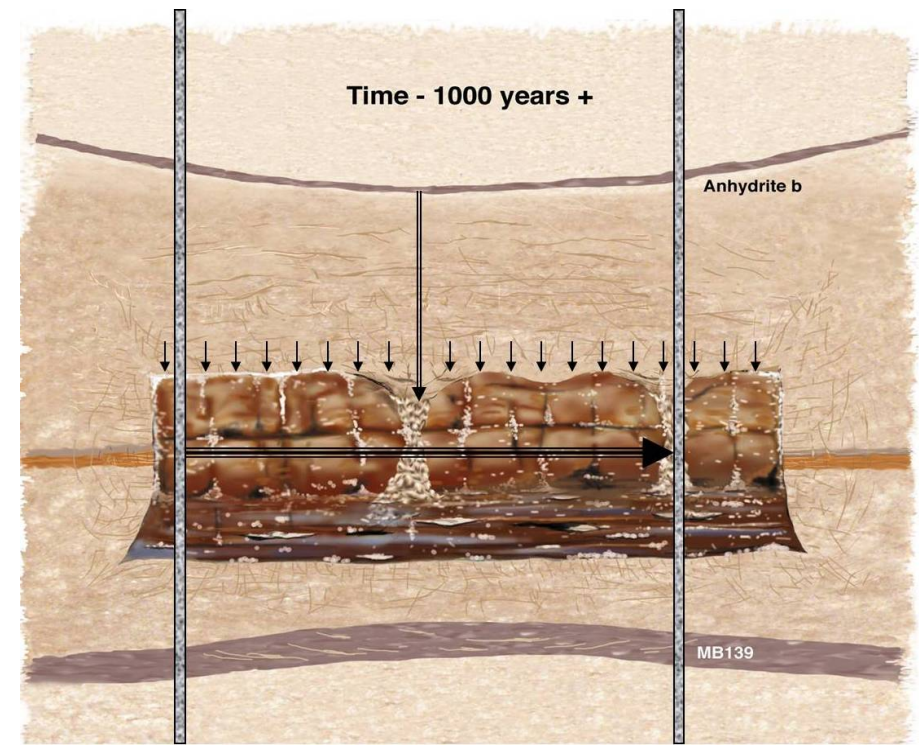
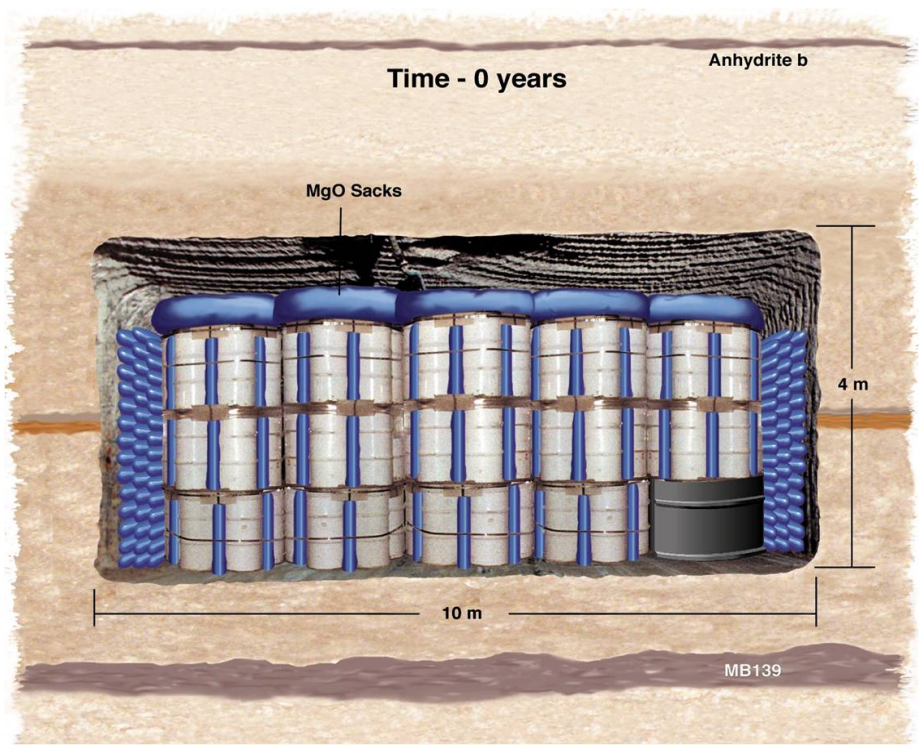
## Remote-handled (RH)

- Small volume
  - ~7,000 m<sup>3</sup> capacity
- Contains short-lived gamma emitters
- Shielding required
- Emplaced in horizontal boreholes in waste room walls





# WIPP Disposal Room: Reliance on Self-Sealing At Time of Closure ... and 1000+ Years Later





# Waste and Actinide Inventory in the WIPP

## Projected Actinide Inventories in the WIPP

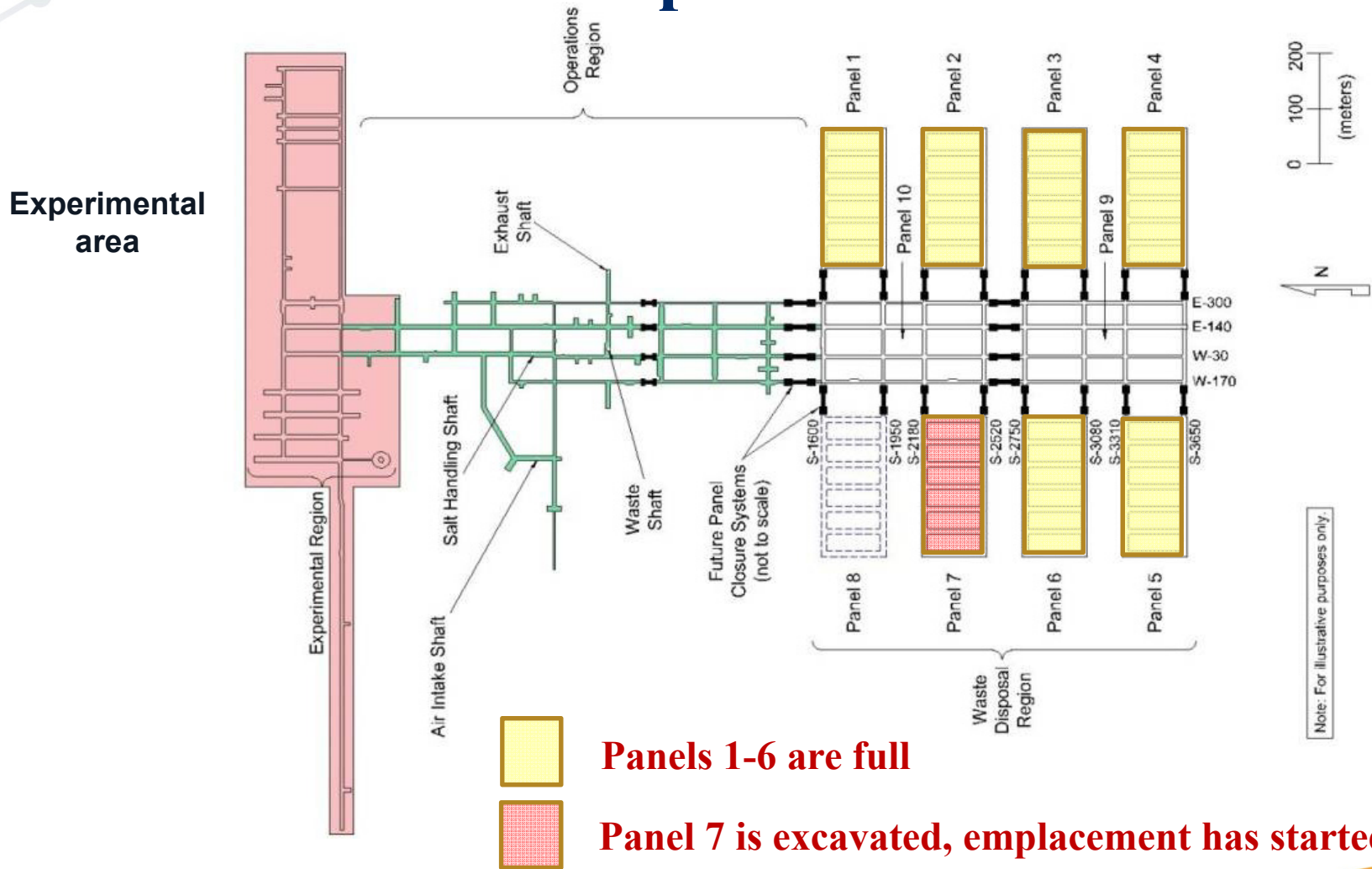
Element	2033 (0 years) Ci (Kg)	12033 (10,000 years) Ci (Kg)
Th	7.04 (1.35E4)	127 (1.35E4)
U	528 (2.26E5)	769 (2.28E5)
Np	23.2 (32.5)	170 (238)
Pu	2.02E6 (1.20 E4)	5.00E5 (9.12E3)
Am	7.05E5 (203)	21.1 (0.0994)
Cm	9.97E3 (0.122)	0.00 (0.00)
Cs	2.35E5 (2.67)	0.00 (0.00)
Sr	2.09E5 (1.51)	0.00 (0.00)

## Projected Inventory of Waste Materials

Material	Total (kg)
Iron-based metals/alloys	4.91E7
Aluminum-based metals/alloys	4.57E5
Lead	8.28E3
<b>Cellulosics</b>	<b>4.65E6</b>
<b>Plastics</b>	<b>9.51E6</b>
<b>Rubber</b>	<b>1.25E6</b>
<b>CPR Total</b>	<b>1.54E7</b>
Cement	1.08E7
MgO	Anticipated to be > 4 x 10 <sup>6</sup>
Organic Ligands (all from Waste)	Acetate: 2.41E4
	Oxalate: 1.85E4
	Citrate: 7.78E3
	EDTA: 3.76E2



# WIPP Panels: Status of Waste Emplacement





# WIPP Record of Operation

## Almost 15 years of operation

**>11,000 shipments received**

**~380,000 loaded drum equivalent containers disposed**

**>86,000 cubic meters of TRU waste disposed (~ 165,000 containers)**

**>12,000,000 loaded miles**

**~7 waste panels mined. Six panels are full, waste emplacement in panel 7 initiated**

**21 storage sites cleaned of legacy TRU waste**

**Emplaced waste contains about 10,000 Kg of Pu**

## Impact on the near environment and personnel

**0 contaminated WIPP personnel**

**Negligible impact on the near-field environment**

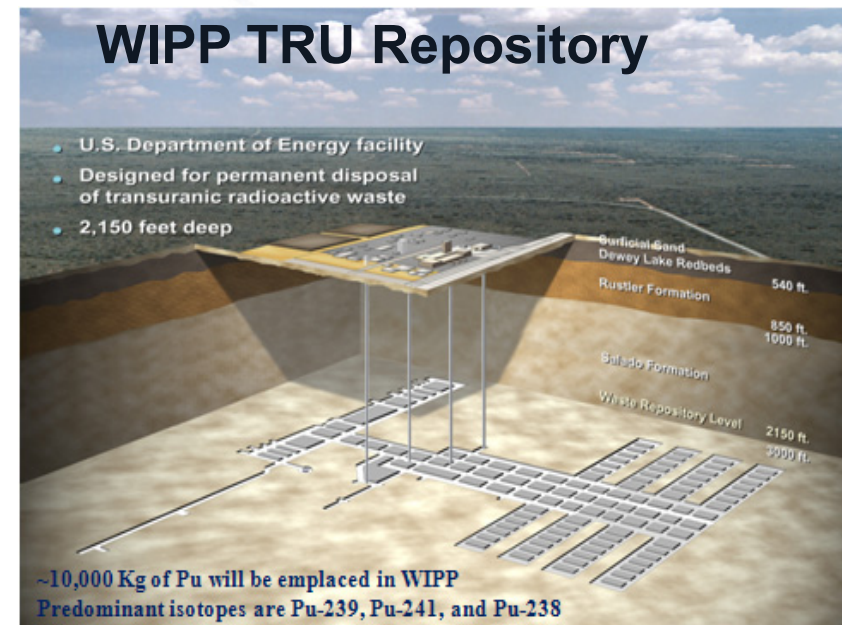
**Over 24 years as NM “Mine Operator of the Year”**



# Motivation and Application of the Research

## WIPP-specific goals

- Support ongoing recertification (next is CRA-2014)
- Support model changes and updates under consideration (2-year program plan)
- Measure needed/planned parameter updates



## Rationale for the work beyond WIPP

- 1) Groundwork for all WIPP repository options
- 2) Possible expansion of Salt-based repositories into HLW



# Approach to Actinide Concentrations in the WIPP

- 1. Assess and track actinide inventory**
  - a) Pu, Am, Cm, U, and Th are potentially important**
  - b) Cm and Np is eliminated as a consideration**
  - c) Available chelating/complexing ligands**
- 2. Assign oxidation state distribution by expert opinion**
  - a) Built-in conservatism**
  - b) Fe/microbially-induced redox environment that is reducing**
  - c) Support with WIPP-specific data**
- 3. Establish effective solution concentration**

**Model/measure actinide solubilities using redox-invariant analogs**

  - a) Account for colloidal contribution by process-specific enhancement factors: intrinsic, bio, inorganic, HA**
  - b) Assign an uncertainty distribution based on literature data review**



# Importance of Actinides and Oxidation States in the WIPP

**Overall Release:** Pu ~ Am >> U > Th >> Np, Cm and fission products

**Oxidation State:** III > IV >> VI > V

- Overall we are concerned with the release Am(III) and Pu(III/IV)
- Base safety case is that the repository stays dry due to self-sealing
- Brine inundation is a low-probability event that drives the potential for dissolved brine release (DBR) from the WIPP





# Assumptions on the Actinide Oxidation- State Distribution in the WIPP

**Oxidizing:**

**U(VI), Pu(IV), Np(V)**

**Reducing:**

**U(IV), Pu(III), Np(IV)**

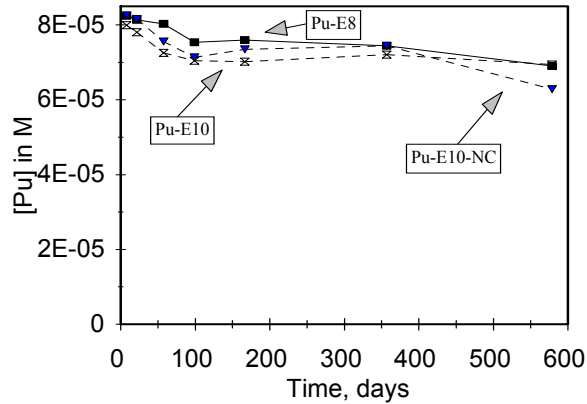
**E<sub>h</sub> independent**

**Th(IV), Am(III),  
Cm(III)**

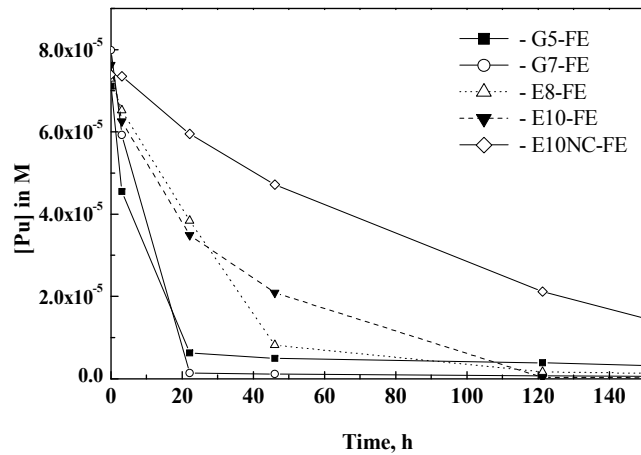
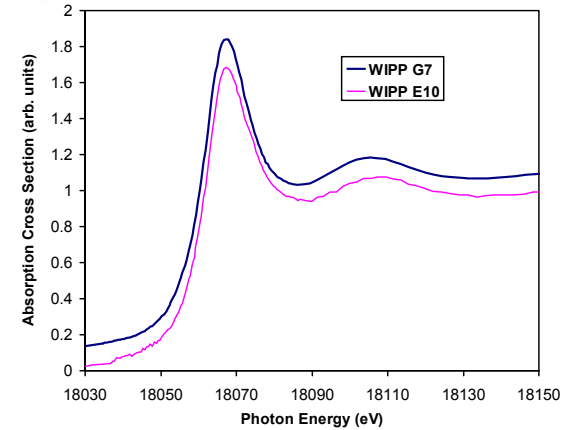
Table 2. Actinide Oxidation State Distribution Assumed in the WIPP Performance Assessment Model					
Actinide	Oxidation State				Speciation Data used in Model Predictions
	III	IV	V	VI	
Thorium		100%			Thorium
Uranium		50%		50%	Thorium for U(IV), 1 mM fixed value for U(VI)
Neptunium		50%	50%		Thorium for Np(IV), neptunium for Np(V)
Plutonium	50%	50%			Americium/neodymium for Pu(III) and thorium for Pu(IV)
Americium	100%				Americium/neodymium
Curium	100%				Americium/neodymium



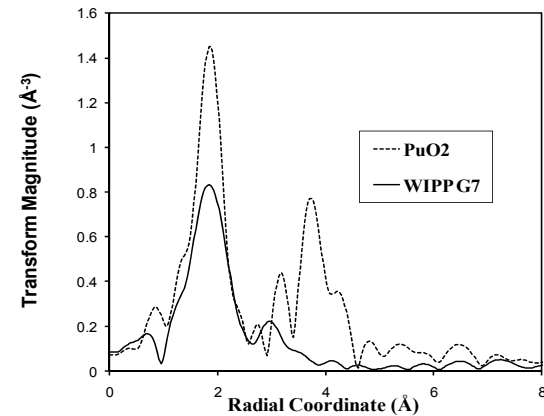
# Plutonium Oxidation State in Brine 1990s Experiments



**No reductant**



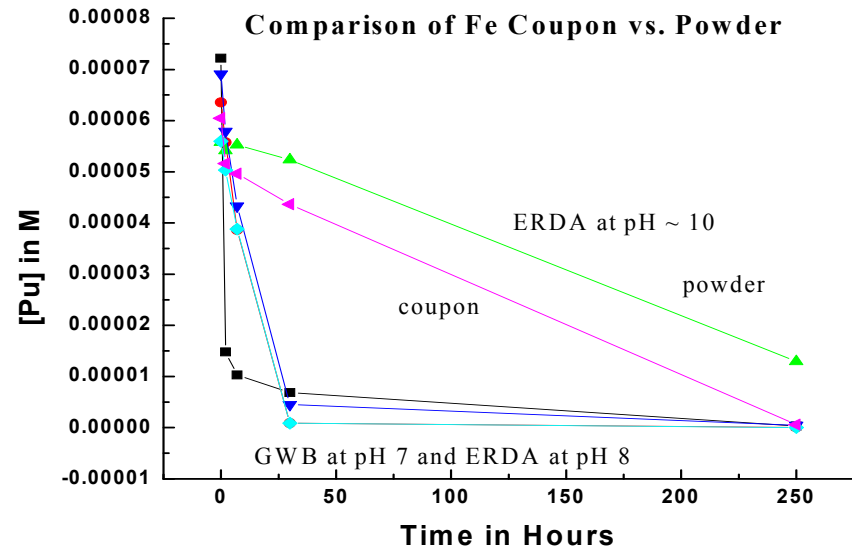
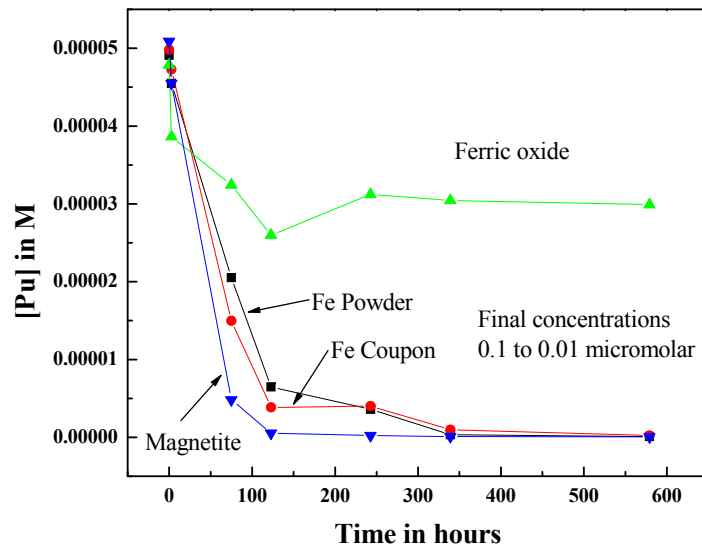
**Fe Added**





# Plutonium Oxidation State Distribution in Brine LANL Experiments

Reduction of Pu(VI) by Iron and Iron Oxides



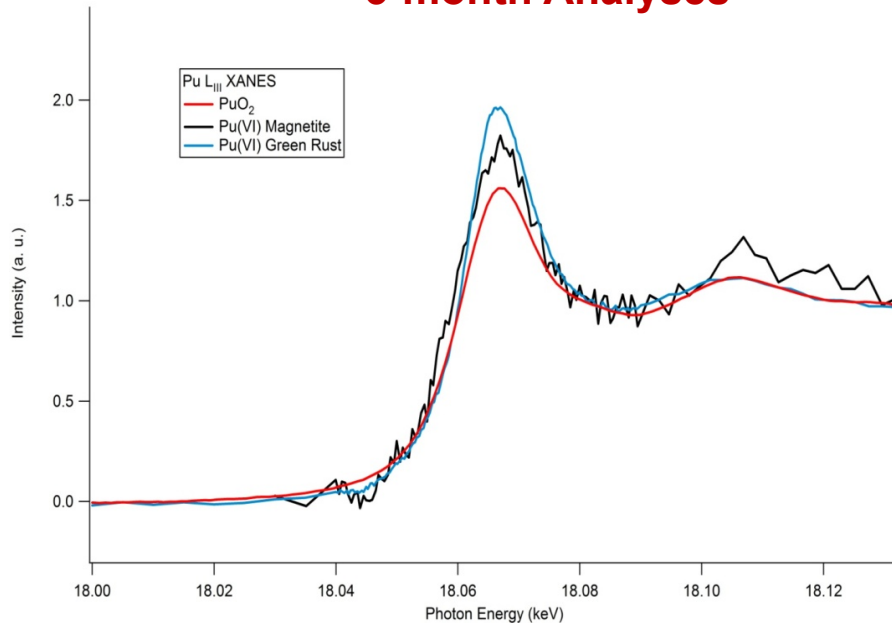
**Pu(V/VI) reduction was always observed when reduced Fe, Fe(0/II), was present**



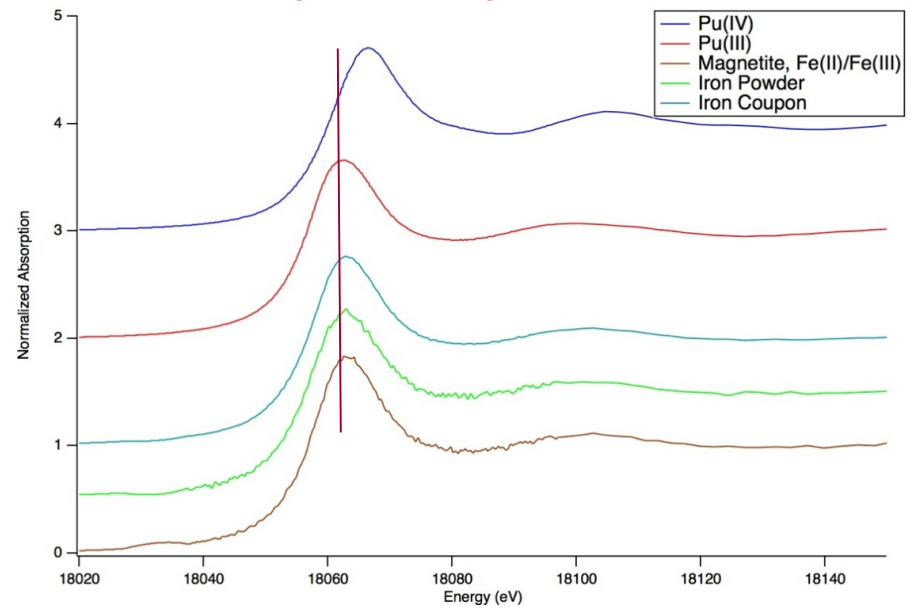
# Pu(V/VI) Reduction by Lower-Valent Fe in Brine

Final Pu oxidation state is mostly Pu(III) and Pu(IV) colloid based on TTA extraction, ~ no extractable Pu(IV)

~ 3-month Analyses



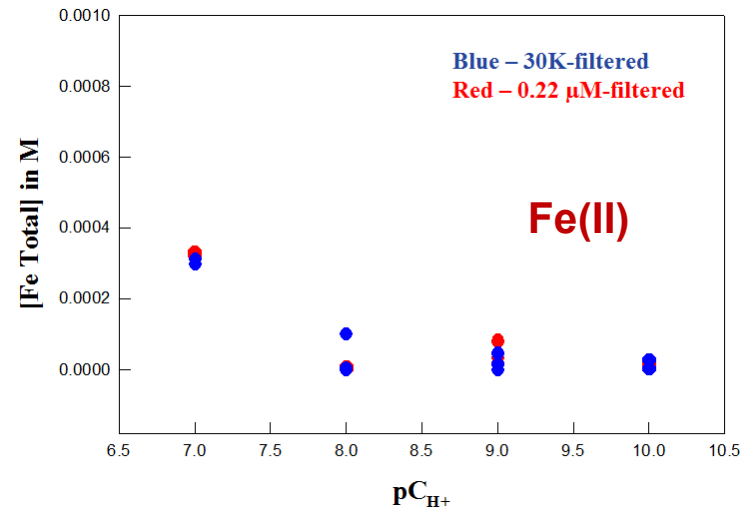
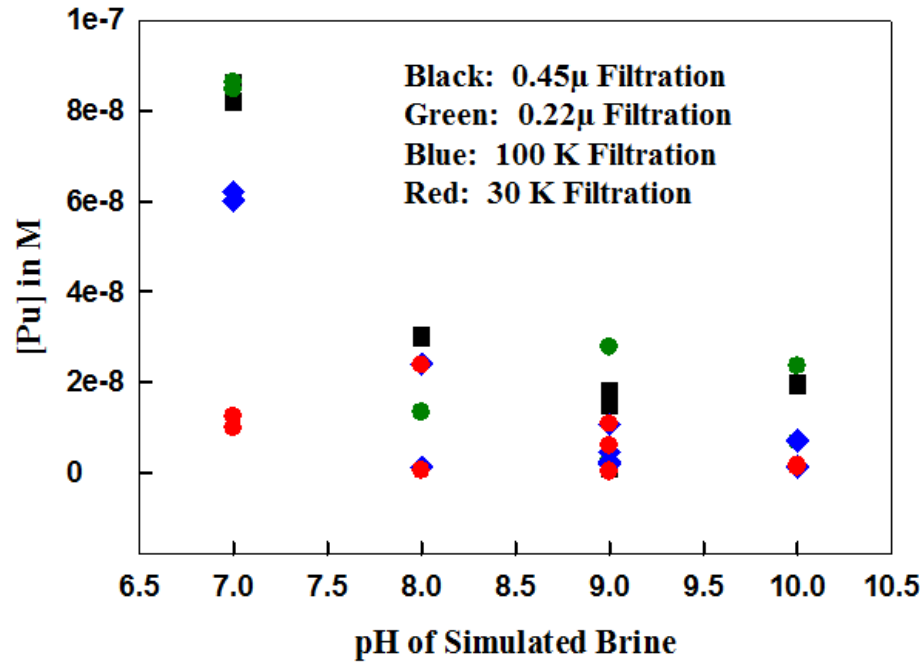
~ 6-year Analyses





# Plutonium (III) Association with Iron Colloids

## Plutonium (III)



Long-term Pu studies with iron show colloidal enhancement well above the >10 nm operational definition of intrinsic plutonium colloids and correlates with the [Fe] present in solution. states



# Summary: Pu Redox in the WIPP

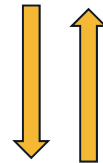
- **Reducing conditions are generated due to anoxic corrosion that leads to Fe<sup>2+</sup> production**
- **Pu(III) predominates in solid and aqueous phase for Pu-242 and the presence of reduced iron – the long-term role of Pu(III) is not understood**
- **Radiolysis and the presence of organics may shift this redox to Pu(IV) even under Fe-dominated anoxic conditions (still under investigation)**

Pu(V/VI)



**Rapid (aqueous)**

Pu(IV)



**Slow (solid phase?)**

Pu(III)



# Actinide Solubility: General Approach

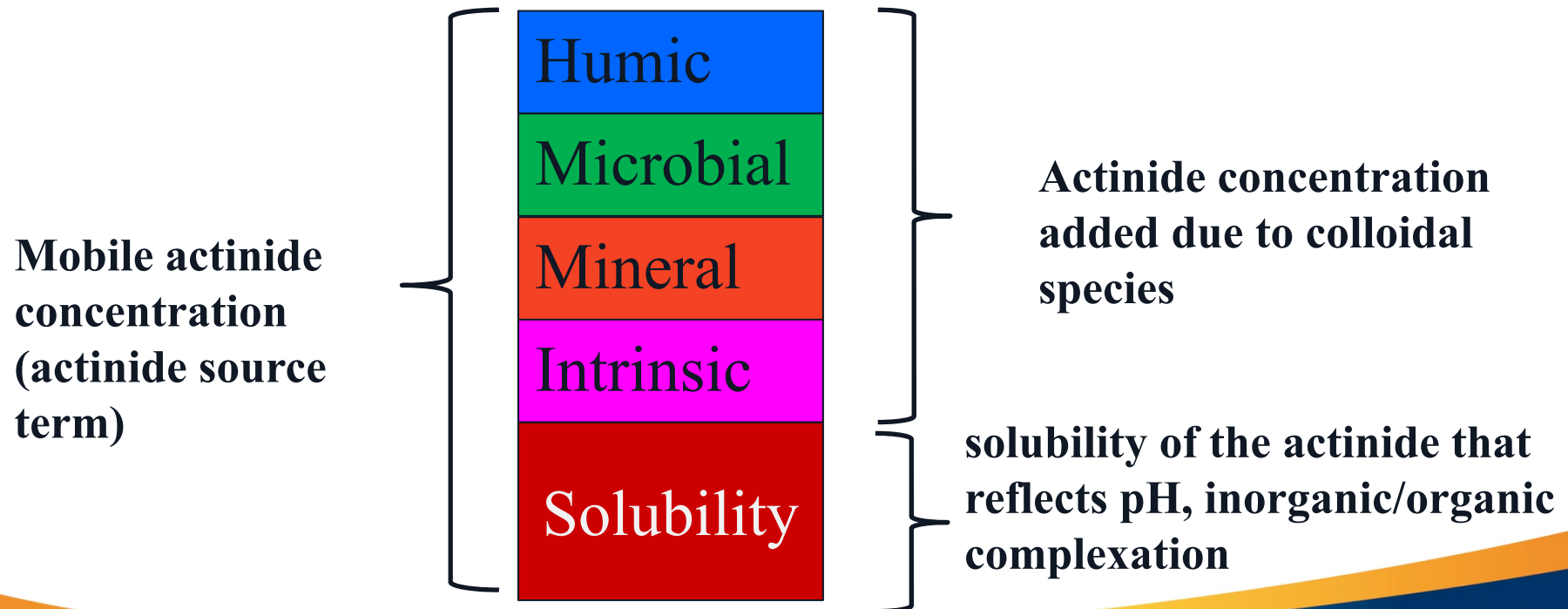
- **Pitzer parameter approach (all brines at  $I > 5M$ )**
- **Use of simplified thermodynamic speciation models that can be defined as conservative**
- **Use of oxidation-state invariant analogs is critical to experimentally assure that the oxidation state distribution is known – this also introduces conservatisms**
- **Simulated brine approach to calculations and experiments (GWB and ERDA-6)**
  - **pH ~ 8.7 to 8.9 (corresponds to  $pC_{H^+} = 9.2-9.4$ )**
  - **Organic complexants based on inventory – only EDTA and Citrate can affect actinide solubilities**
  - **Carbonate is fixed by MgO buffering, expected is ~ 10 mM maximum**
- **Reliance on modeling first, confirmation by experiment**



# WIPP Model (~1995) for the Mobile Actinide Concentration

**In the WIPP concept, colloidal species contribute to the source term in dissolved brine release (DBR) release scenarios**

- Colloidal transport is not a significant issue
- Structure and physical properties are not important







# Completed/Ongoing LANL Actinide Solubility/Speciation Studies

**An(III) Solubility Studies: Nd(III) and Am(III); Pu(III) stability**

**An(IV) Solubility Studies: Th(IV) and Np(IV)**

**An(VI) Solubility Studies: U(VI)**

**Colloid Studies: intrinsic colloids, mineral colloids**

**Microbial interactions studies: Bioreduction and Biosorption**

**Selected Example**



# Thorium Solubility Studies: Experimental Approach

## Brine Systems:

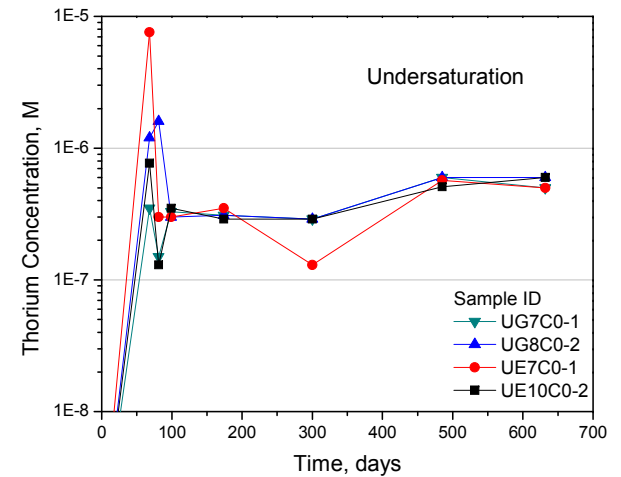
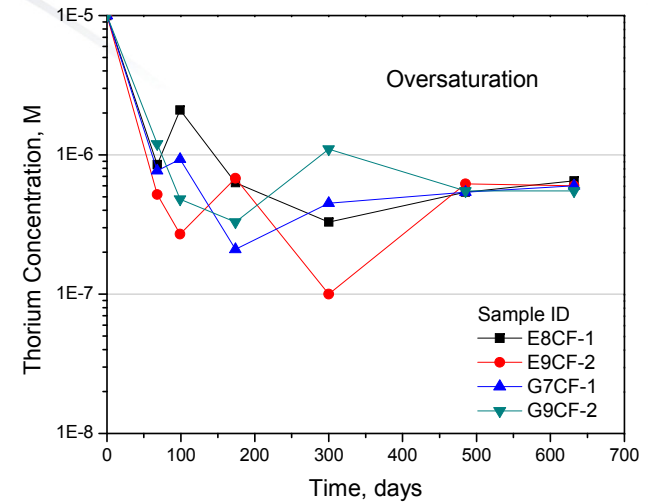
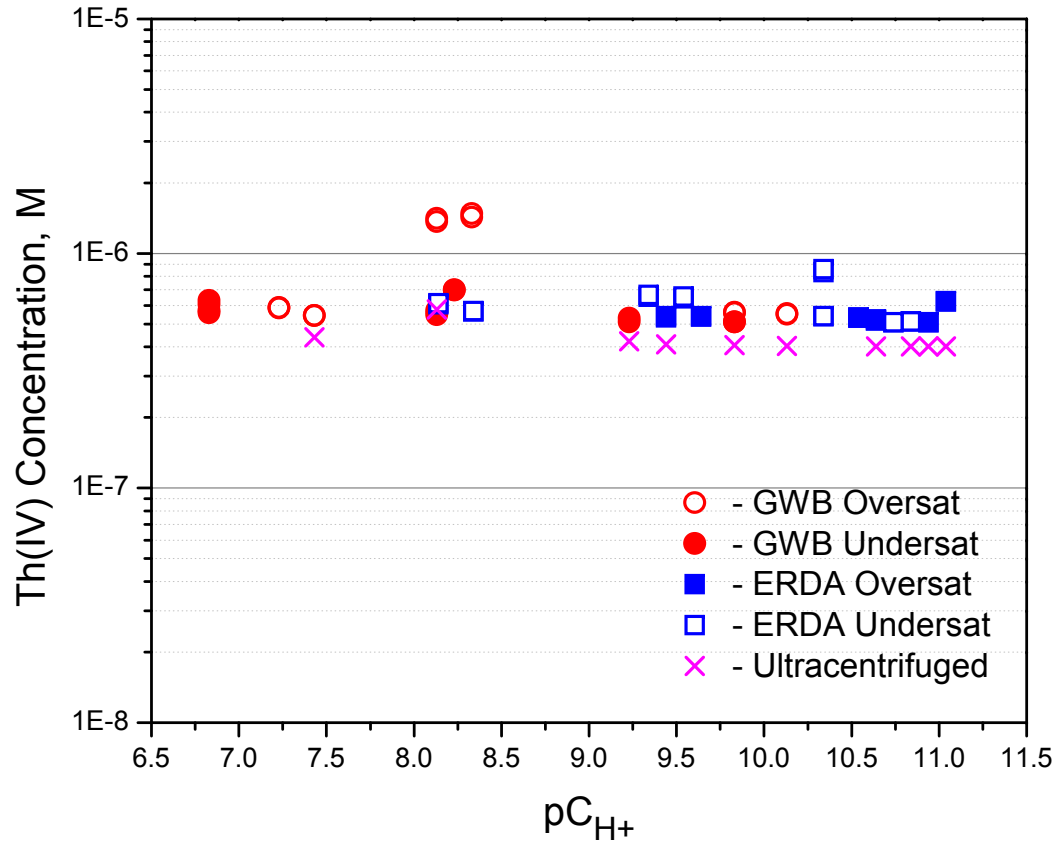
- **GWB and ERDA-6 Brine – Bracketing approach**
- **Anoxic conditions in Glovebox**
- **pH range of 7-12**

## General Approach:

- **Long-term (approaching 3 years) data**
- **Routinely filtered to 100K MWCO (~20 nm)**
- **ICP-MS analysis to measure concentration**
- **Effects of organics and carbonate evaluated**



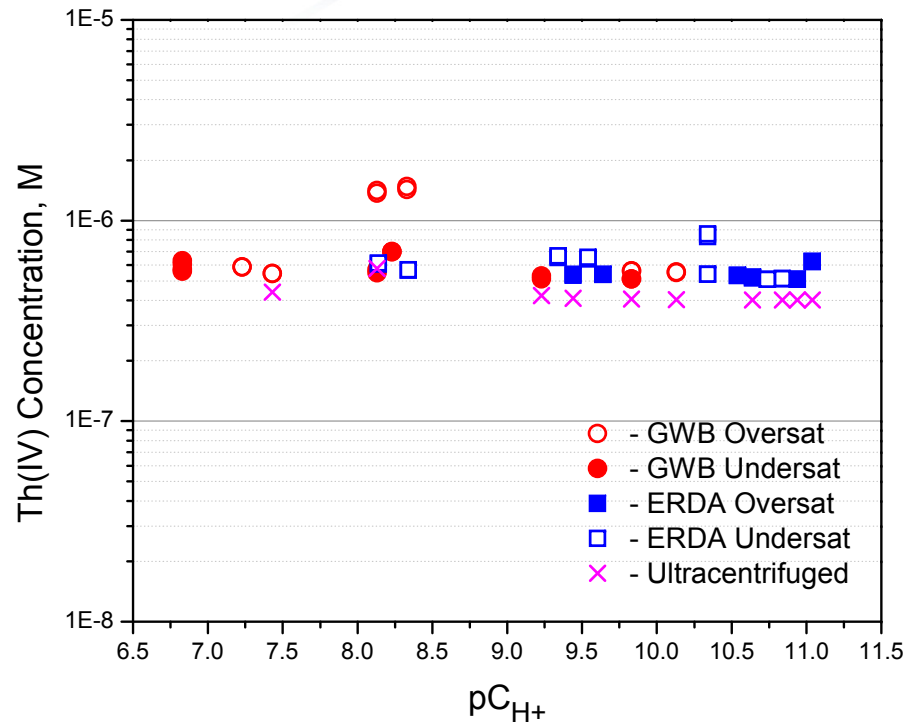
# An(IV): Solubility of Thorium in WIPP



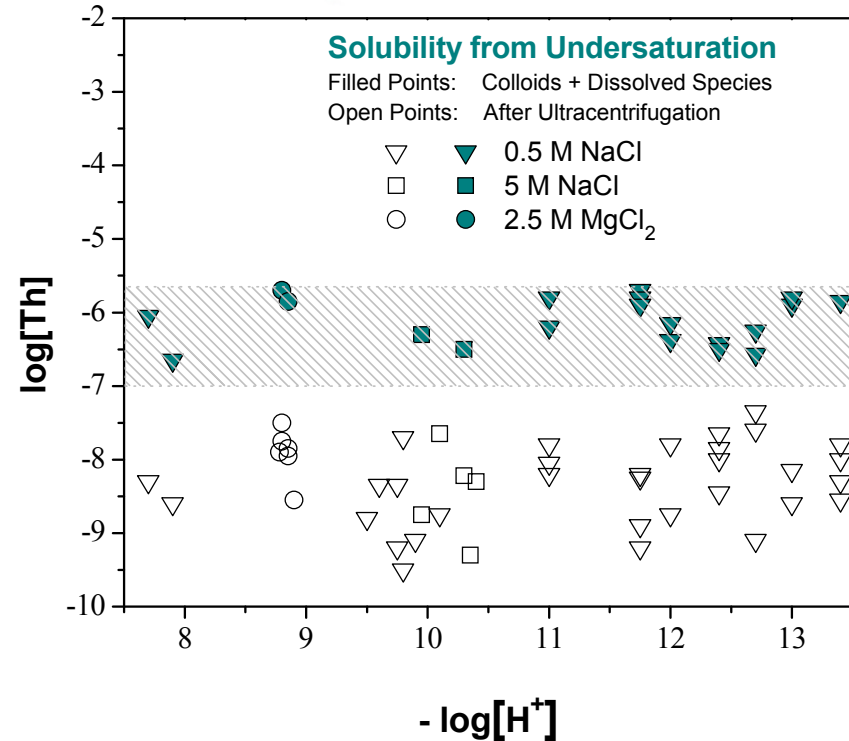


# Intrinsic Colloids: Thorium Data

## LANL/WIPP



## INE/Germany



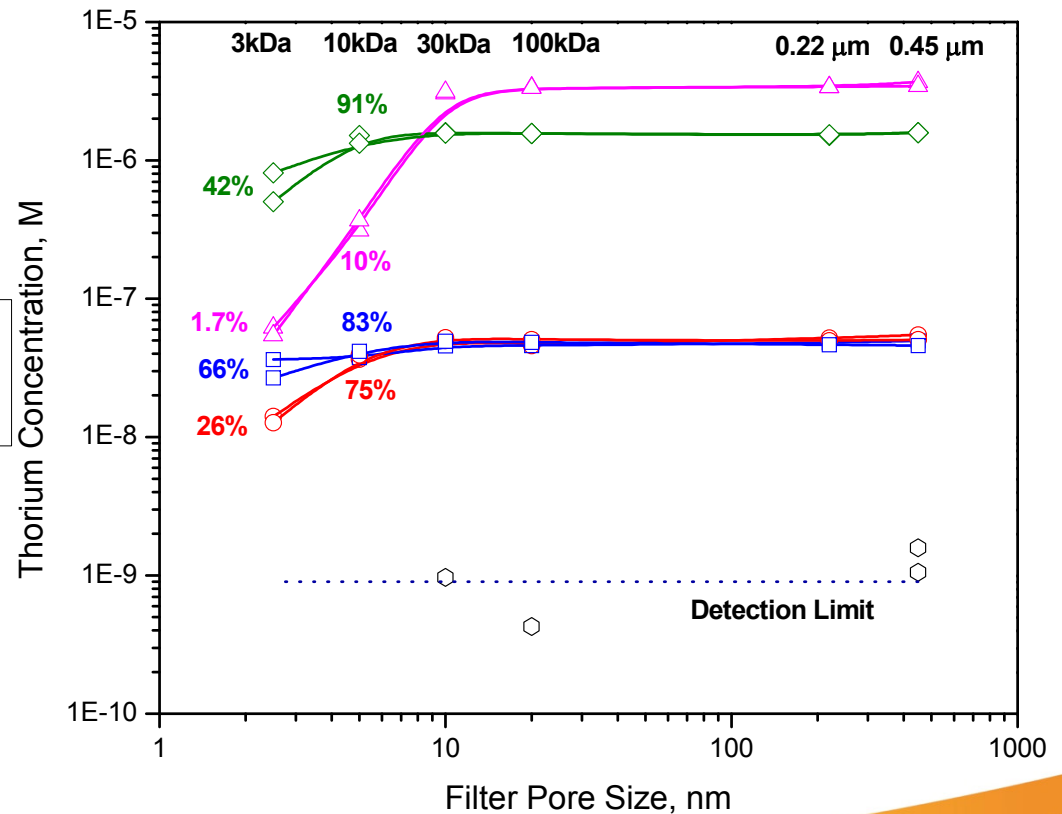
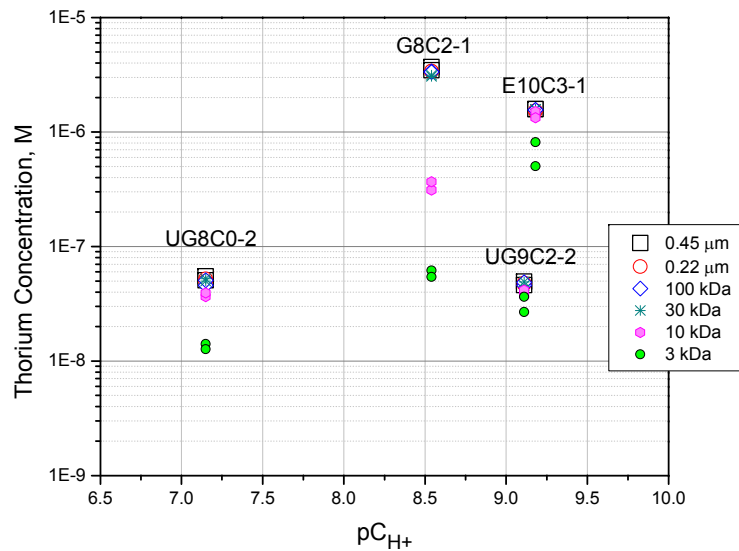
### Why are these different?

- 1) Brine Composition and degree of saturation – Mg colloidal content
- 2) Time – colloidal fraction decreases with time



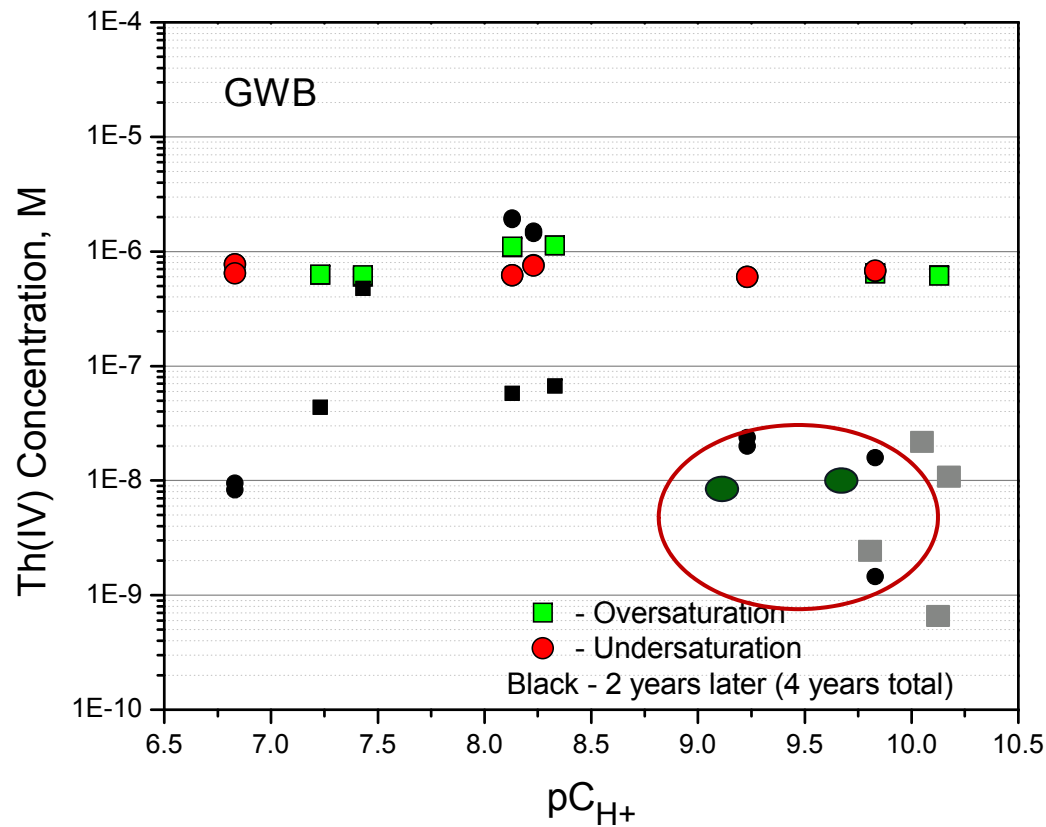
# Effect of Filtration on Thorium Concentrations

~ no effect to 10 nm, 6 nm data matches ultracentrifugation results





# Results of Th(IV) solubility measured after 4 years of equilibration

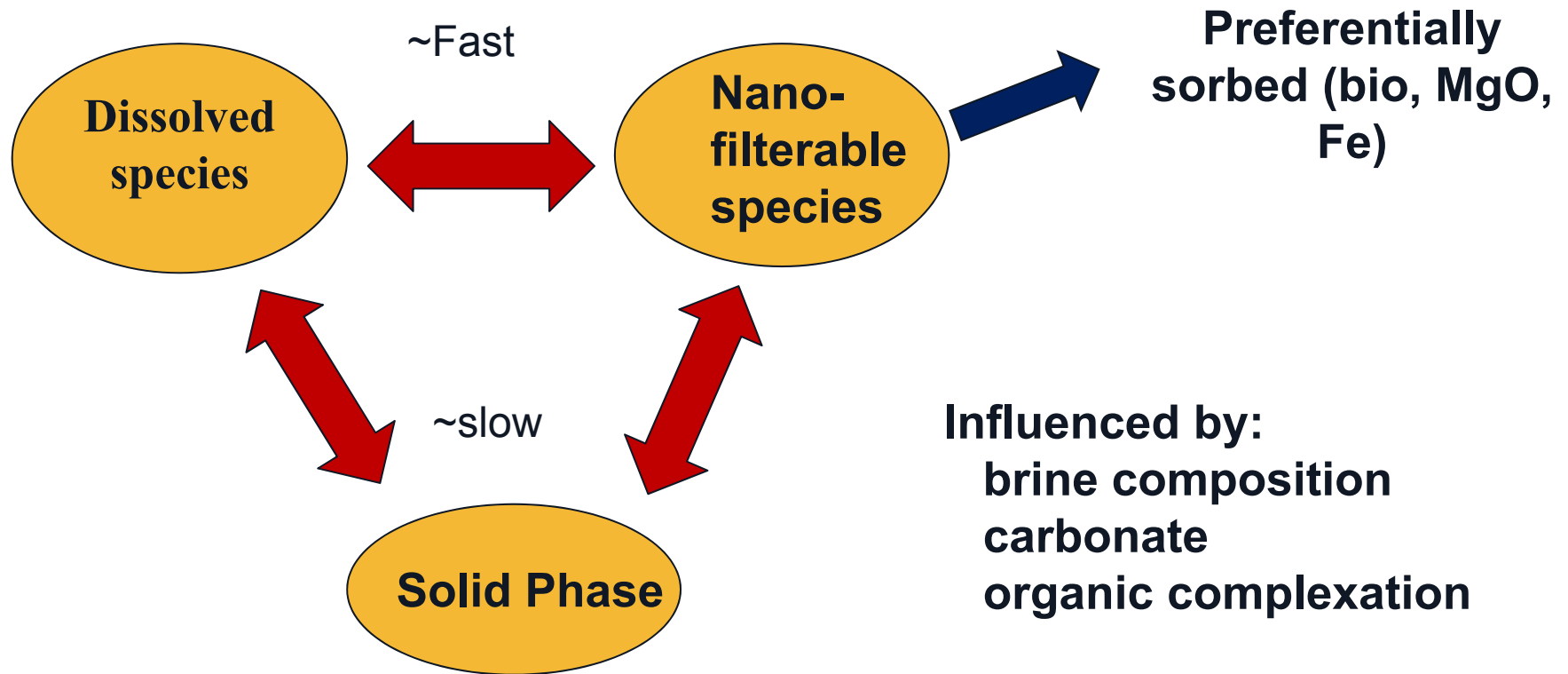


■ Altmaier et al., 2004 – 5 M NaCl ultracentrifugation data

● Biosorption experiments with *Chromohalobacter sp.*



# Summary of Observations: Thorium Brine System



*Also proposed in Altmaier, Neck, Fanghanel. Radiochimica Acta 92(9-11), 537-543 (2004).*



# Oxidation-State-Specific Actinide Concentrations

<b>Calculated Actinide Solubility: Historical Trends</b>			
<b>Actinide Oxidation State, and Brine</b>	<b>CRA-2004 PABC (M)</b>	<b>CRA-2009 PABC (M)</b>	<b>CRA-2014 PA (M)</b>
<b>III, GWB</b>	$3.87 \times 10^{-7}$	$1.66 \times 10^{-6}$	$2.59 \times 10^{-6}$
<b>III, ERDA-6</b>	$2.88 \times 10^{-7}$	$1.51 \times 10^{-6}$	$1.48 \times 10^{-6}$
<b>IV, GWB</b>	$5.64 \times 10^{-8}$	$5.63 \times 10^{-8}$	$6.05 \times 10^{-8}$
<b>IV, ERDA-6</b>	$6.79 \times 10^{-8}$	$6.98 \times 10^{-8}$	$7.02 \times 10^{-8}$
<b>V, GWB</b>	$3.55 \times 10^{-7}$	$3.90 \times 10^{-7}$	$2.77 \times 10^{-7}$
<b>V, ERDA-6</b>	$8.24 \times 10^{-7}$	$8.75 \times 10^{-7}$	$8.76 \times 10^{-7}$





# Summary of Observations

- **WIPP, overall, continues to have success, alternative options in salt are under consideration**
- **Establishing a favorable redox environment in the near-field is critical to a successful remediation or long-term repository strategy for actinides:**
  - **Under reducing anoxic conditions higher-valent actinides are not expected to contribute to long-term subsurface migration**
  - **Lower solubility and immobility predominate**
- **WIPP addresses actinide redox through expert judgment that defined conservative oxidation-state distributions**
  - **WIPP-specific Pu-Fe interaction studies show reduced Pu oxidation states are established**
  - **Good success in showing that indigenous microorganisms impact redox in similar ways as soil bacteria – but we are not there yet**



## Summary of Observations – cont.

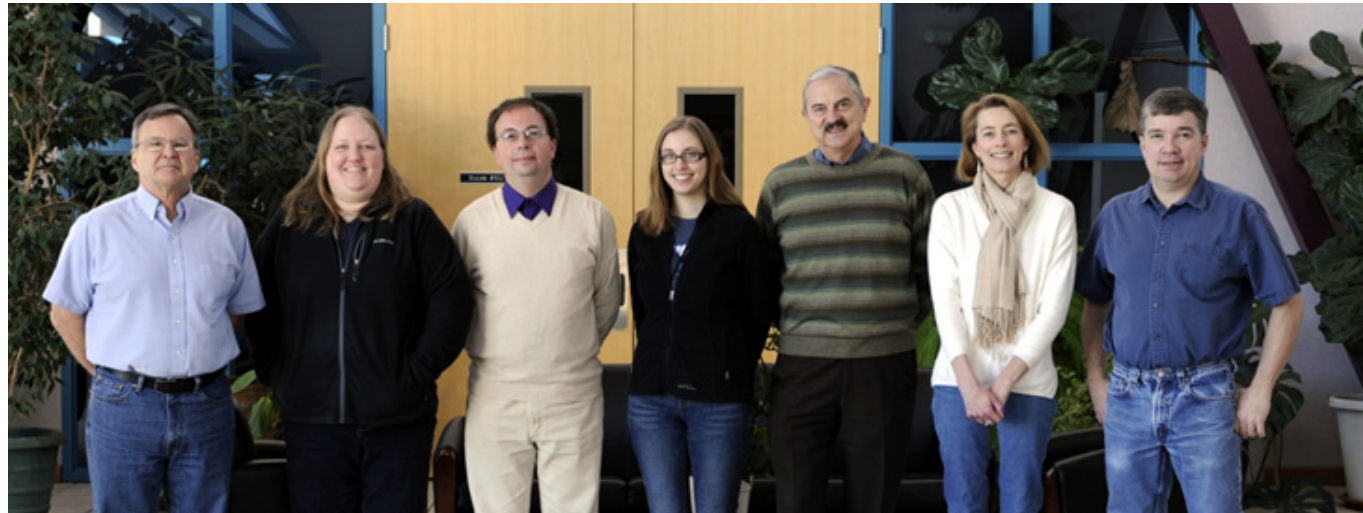
- **Correlation was observed between the  $E_h$  measurements, Fe(II/III) ratios and Pu(III/IV) ratios**
  - $E_h$  in iron dominated system is likely meaningful
  - Need Pitzer data to properly model the system
  - Role of Pu(III) is not fully understood and is the focus of continued research
- **Measured WIPP-specific solubilities are consistent with, but slightly higher than model predictions**
  - Nd(III) solubility slightly impacted by borate, significant impact by organics
  - Th(IV) shows little colloid formation and is largely unaffected by organics, borate, carbonate and pH under WIPP conditions



**Thanks for your attention**

## Acknowledgements

**LANL  
ACRSP  
Team**



- **Russ Patterson: DOE program manager for this research**
- **Synchrotron-based studies: Advanced Photon Source facility at Argonne - special thanks to Dan Olive and Jeff Terry (IIT)**
- **Research is supported by the Waste Isolation Pilot Plant, Department of Energy, Carlsbad Field Office**